which are adequate to protect the health and safety of the public, and minimize danger to life or property.

A copy of the final supporting statement may be viewed free of charge at the NRC Public Document Room, 2120 L Street, NW (lower level), Washington, DC. OMB clearance requests are available at the NRC worldwide web site (http://www.nrc.gov/NRC/PUBLIC/OMB/index.html). The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions should be directed to the OMB reviewer listed below by July 16, 1999. Comments received after this date will be considered if it is practical to do so, but assurance of consideration cannot be given to comments received after this date.

Erik Godwin, Office of Information and Regulatory Affairs (3150–0120), NEOB–10202, Office of Management and Budget, Washington, DC 20503.

Comments can also be submitted by telephone at (202) 395–3087.

The NRC Clearance Officer is Brenda Jo. Shelton, 301–415–7233.

Dated at Rockville, Maryland, this 10th day of June 1999.

For the Nuclear Regulatory Commission.

Brenda Jo. Shelton,

NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 99–15242 Filed 6–15–99; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. STN 50–528, STN 50–529, and STN 50–530]

Arizona Public Service Company; Palo Verde Nuclear Generating Station, Units 1, 2, and 3; Notice of Withdrawal of Application for Amendments to Facility Operating Licenses

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Arizona Public Service Company (the licensee) to withdraw its November 6, 1996, application for proposed amendments to Facility Operating Licenses Nos. NPF–41, NPF–51, and NPF–74, for the Palo Verde Nuclear Generating Station (Palo Verde), Units 1, 2, and 3, located in Maricopa County, Arizona.

The proposed amendments would have revised the facility technical specifications to provide a method to respond to a sustained, degraded switchyard voltage condition.

The Commission had previously issued a Notice of Consideration of Issuance of Amendments published in the **Federal Register** on January 2, 1997 (62 FR 123). However, by letter dated December 16, 1998, the licensee withdrew the proposed change.

For further details with respect to this action, see the application for amendments dated November 6, 1996, and the licensee's letter dated December 16, 1998, which withdrew the application for license amendments. The above documents are available for public inspection—2 -at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Phoenix Public Library, 1221 N. Central Avenue, Phoenix, Arizona 85004.

Dated at Rockville, Maryland, this 7th day of June 1999.

For the Nuclear Regulatory Commission.

Mel B. Fields,

Project Manager, Section 2, Project Directorate IV & Decommissioning Division of Licensing Project Management Office of Nuclear Reactor Regulation

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. STN 50–454, STN 50–455, STN 50–456 and STN 50–457]

Commonwealth Edison Company; Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. NPF–37 and NPF–66 issued to the Commonwealth Edison Company (ComEd, the licensee) for operation of Byron Station, Unit Nos. 1 and 2, respectively, located in Ogle County, Illinois, and Facility Operating License Nos. NPF–72 and NPF–77 issued to ComEd for the operation of Braidwood Station, Unit Nos. 1 and 2, respectively, located in Will County, Illinois.

The proposed amendments would change the Technical Specifications to support a plant modification to install new storage racks for fuel in the spent fuel pools (SFP). As part of the modification, the total capacity of the SFP at each station is being increased from 2,870 assemblies to 2,984 assemblies.

Before issuance of the proposed license amendments, the Commission will have made findings as required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendments requested involve no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendments would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

The proposed Technical Specifications (TS) changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

During the installation of the new Holtec spent fuel pool storage racks, both Holtec and the existing Joseph Oat spent fuel pool storage racks will be in the spent fuel pool at the same time. This interim arrangement will not increase the probability or consequences of an accident previously evaluated. The criticality analysis for the Joseph Oat spent fuel pool storage racks states that should a spent fuel pool water temperature change accident or a fuel assembly misload accident occur in the Region 1, Region 2, or failed fuel storage cells, k_{eff} will be maintained less than or equal to 0.95 due to the presence of at least 550 ppm (no fuel handling) or 1650 ppm (during fuel handling) of soluble boron in the spent fuel pool water. These assumptions are more conservative than the requirements stated in the criticality analysis for the Holtec spent fuel pool storage racks which only requires 220 ppm boron to maintain k_{eff} less than or equal to 0.95 during the worst case fuel assembly misload accident. The new Holtec racks have a superior neutron attenuation capability due to their improved design. The requirement of 2000 ppm boron will be maintained during the entire change out process, therefore, ensuring that k_{eff} will remain less than or equal to 0.95. At the completion of installation, only Holtec spent fuel pool storage racks will be in the spent fuel pool.

The previously evaluated Byron and Braidwood Stations accidents relative to spent fuel storage are discussed in the Updated Final Safety Analysis Report (UFSAR) Section 15.7.4, ''Fuel Handling Accidents,'' and UFSAR Section 15.7.5, ''Spent Fuel Cask Drop Accident.'' These accidents were considered for the new Holtec spent fuel pool racks and are listed below.

a. Spent fuel assembly dropped onto the spent fuel pool floor.