

for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendments dated March 22, 1999, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Byron Public Library District, 109 N. Franklin, P.O. Box 434, Byron, Illinois 61010 (for Byron) and the Wilmington Public Library, 201 S. Kankakee Street, Wilmington, Illinois 60481 (for Braidwood).

Dated at Rockville, Maryland, this 23rd day of March 1999.

For the Nuclear Regulatory Commission.
John B. Hickman,
*Project Manager, Project Directorate III-2,
 Division of Licensing Project Management,
 Office of Nuclear Reactor Regulation.*
 [FR Doc. 99-7601 Filed 3-26-99; 8:45 am]
 BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 72-20]

Department Of Energy, Idaho Operations Office Notice of Issuance of Materials License SNM-2508 for TMI-2 Independent Spent Fuel Storage Installation

The U.S. Nuclear Regulatory Commission (NRC or the Commission) has issued a Materials License under the provisions of Title 10 of the Code of Federal Regulations, Part 72 (10 CFR Part 72), to the Department of Energy, Idaho Operations Office (DOE-ID), authorizing receipt and storage of spent fuel in an independent spent fuel storage installation (ISFSI) located at the Idaho National Engineering and Environmental Laboratory (INEEL), within the Idaho Nuclear Technology and Engineering Center (INTEC) site in Scoville, Idaho, as described in its application dated October 31, 1996, and Safety Analysis Report (SAR).

The function of the ISFSI is to provide interim storage of radioactive material from the Three Mile Island Unit 2 (TMI-2) reactor core damaged by the March 28, 1979, reactor accident, including the remains of 177 Babcock and Wilcox 15x15 fuel assemblies, 61 control rod assemblies, and miscellaneous

irradiated core and core basket material. The material is contained within 265 fuel canisters, 12 knockout canisters, and 67 filter canisters which are used to confine the TMI-2 core debris in the absence of intact fuel assembly cladding. The cask that is authorized for use is the NUHOMS-12T designed by Transnuclear West, Inc. The license for an ISFSI under 10 CFR Part 72 is issued for 20 years, but the licensee may seek to renew the license, if necessary, prior to its expiration.

The Commission's Office of Nuclear Material Safety and Safeguards (NMSS) has completed its environmental, safeguards, and safety reviews in support of issuance of this license.

Following receipt of the application dated October 31, 1996, a "Notice of Consideration of Issuance of a Materials License for the Storage of Spent Fuel and Notice of Opportunity for a Hearing" was published in the **Federal Register** on January 13, 1997 (62 FR 1782). The "Final Environmental Impact Statement (FEIS) Related to the Construction and Operation of the TMI-2 Independent Spent Fuel Storage Installation," NUREG-1626, was issued and noticed in the **Federal Register** (63 FR 13077) on March 17, 1998, in accordance with 10 CFR Part 51. The scope of the FEIS included the construction and operation of an ISFSI on the INEEL site.

The staff has completed its safety review of the TMI-2 ISFSI site application and SAR. Materials License SNM-2508 and the NRC staff's "Safety Evaluation Report for the TMI-2 Independent Spent Fuel Storage Installation" were issued on March 19, 1999. Materials License SNM-2508, the staff's Environmental Impact Statement, Safety Evaluation Report, and other documents related to this action are available for public inspection and for copying for a fee at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC 20555, and at the Local Public Document Room at the INEEL Technical Library, 1776 Science Center Drive, Idaho Falls, ID 83402.

Dated at Rockville, Maryland, this 19th day of March 1999.

For the Nuclear Regulatory Commission.
E. William Brach,
*Director, Spent Fuel Project Office, Office of
 Nuclear Material Safety and Safeguards.*
 [FR Doc. 99-7600 Filed 3-26-99; 8:45 am]
 BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-275 and 50-323]

Pacific Gas and Electric Company; Notice of Consideration of Issuance of Amendments to Facility Operating Licenses and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. NPF-80 and 82, issued to Pacific Gas and Electric Company (PG&E or the licensee), for operation of the Diablo Canyon Power Plant, Units 1 and 2 (DCPP), located in San Luis Obispo County, California.

The initial notice of consideration of issuance of amendment to facility operating license and opportunity for hearing was originally published in the **Federal Register** (63 FR 55152) on October 14, 1998. The information included in the supplemental letters indicates that the original notice, that included 13 proposed beyond-scope issues (BSIs) to the Improved Technical Specifications (ITS) conversion, needs to be expanded (to add 15 new BSIs) and revised (to delete 8 previous BSIs) to include a total of 20 BSIs. This notice supersedes the previous notice.

The proposed amendment, requested by the licensee in a letter dated June 2, 1997, as supplemented by letters dated January 9, June 25, August 5, August 28, September 25, October 16, October 23, November 25, December 4, December 17, and December 30, 1998, and February 24 and March 10, 1999, would represent a full conversion from the current Technical Specifications (CTS) to a set of ITS based on NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Revision 1, dated April 1995. NUREG-1431 has been developed by the Commission's staff through working groups composed of both NRC staff members and industry representatives, and has been endorsed by the staff as part of an industry-wide initiative to standardize and improve the Technical Specifications (TSs) for nuclear power plants. As part of this submittal, the licensee has applied the criteria contained in the Commission's "Final Policy Statement on Technical Specification Improvements for Nuclear Power Reactors (Final Policy Statement)," published in the **Federal Register** on July 22, 1993 (58 FR 39132), to the CTS, and, using NUREG-1431 as a basis, proposed an ITS for DCPP. The criteria in the Final Policy Statement were subsequently added to 10 CFR 50.36, "Technical Specifications," in a